Vice President Farley Project Southern Nuclear Operating Company P.O. Box 1295 Birmingham, Alabama 35201 Tel 205.992.5131

September 28, 1998



Docket No.:

50-348

10 CFR 50.73

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555

Joseph M. Farley Nuclear Plant - Unit 1
Licensee Event Report No. 98-004-00
Reactor Protection System Card Failure Caused Turbine Trip
and Consequent Reactor Trip

Ladies and Gentlemen:

Joseph M. Farley Nuclear Plant - Unit 1 Licensee Event Report No. 98-004-00 is being submitted in accordance with 10 CFR 50.73(a)(2)(iv). There are no NRC commitments in the Licensee Event Report.

If you have any questions, please advise.

Respectfully submitted,

Dave Morey

EWC/maf rxsycd.doc

Enclosure

: Mr. L. A. Reyes, Region II Administrator

Mr. J. I. Zimmerman, NRR Project Manager

Mr. T. P. Johnson, Plant Sr. Resident Inspector

IE27/1

9810070019 980928 PDR ADDCK 05000348 S PDR 113

NRC	FORM	366
(6-19	98)	

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

APPROVED BY OMB NO. 3150-0104 EXPIRES 06/30/2001

Estimated burden per response to comply with this mandatory information collection request: 50 hrs. Reported lessons learned are incorporated into the licensing process and fed back to industry. Forward comments regarding burden estimate to the Records Management Branch (T-6 F33), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to the Paperwork Reduction Project (3150-0104), Office of Management and Budget, Washington, DC 20503. If an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

FACILITY NAME (1)

Joseph M. Farley Nuclear Plant- Unit 1

05000348

PAGE (3) 1 OF 3

TITLE (4)

Reactor Protection System Card Failure Caused Turbine Trip and Consequent Reactor Trip

EVE	NT DA	TE (5)	U	ER NU	MRE	RIG	il L	REP	ORT DA	TE (7)	OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQU	JENTI IMBEI		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME		D	OCKET NUMBER		
09	09	1998	1998	0	0	4	0	09	28	1998	FAC	CILITY NAME	DOCKET NUMBER			
OPERA MODE			ТН	IS RE	PORT	r IS	SUBMITTE	D PURSU	ANT TO	THE REO	IUDE	MENTS OF 10 CFR 8: (0	Check on	ne or more) (11)		
	131	1	20.22	201(b)				20.2203			T	50.73(a)(2)(i)		50.73(a)(2)(viii)		
POW		100	20.22					20.2203	(a)(2)(v		I					
	ER	100		203(a)	(1)				(a)(2)(v (a)(3)(i)			50.73(a)(2)(i)		50.73(a)(2)(viii)		
POW	ER	100	20.22	203(a) 203(a)	(1) (2)(i)			20.2203	(a)(2)(v (a)(3)(i) (a)(3)(ii)		X	50.73(a)(2)(i) 50.73(a)(2)(ii)		50.73(a)(2)(viii) 50.73(a)(2)(x)		
POW	ER	100	20.22	203(a) 203(a) 203(a)	(1) (2)(i) (2)(ii)		20.2203	(a)(2)(v (a)(3)(i) (a)(3)(ii) (a)(4)			50.73(a)(2)(i) 50.73(a)(2)(ii) 50.73(a)(2)(iii)	S	50.73(a)(2)(viii) 50.73(a)(2)(x) 73.71		

LICENSEE CONTACT FOR THIS LER (12)
TELEPHONE NUMBER (Include Area Code)

L.M. Stinson, General Manager- Nuclear Plant

(334) 899-5156

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	ENI	CAUSE	SYSTEM	COMPONENT	MANUFA	CTURER	REPORTABLE TO EPIX
X	JC	RLY	W120	Y							
	s	UPPLEMENTAL	REPORT EXPECT	ED (14)			EXF	PECTED	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE).					(10	EXF	PECTED	MONTH	DAY	

ABSTRACT

NAME

On September 9, 1998, while operating in Mode 1 at 100% power, the Unit 1 reactor automatically tripped due to a main turbine trip. Additionally, both steam generator feed pumps (SGFPs) automatically tripped and a main feedwater isolation occurred. The cause of the trip was failure of a circuit card in solid state protection system (SSPS) train B. Failure of this card resulted in a spurious hi-hi steam generator water level signal. This condition directly caused a main turbine trip, both SGFP trips, and a main feedwater isolation. The main turbine trip resulted in an automatic reactor trip. The specific failure mechanism for this card is under investigation by the supplier.

All other systems functioned as designed and plant response to the transient was as expected.

The failed card was replaced. Unit 1 returned to power operation on September 10, 1998.

NRC FORM 366A (6-1998) U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET (2)		LER N	NUME	BER (6	5)		PAGE	(3)
		YEAR	SEQUENTIAL NUMBER			REVISION NUMBER			
Joseph M. Farley Nuclear Plant Unit 1	05000348	1998	0	0	4	00	2	OF	3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

Plant and System Identification

Westinghouse -- Pressurized Water Reactor Energy Industry Identification System codes are identified in the text as [XX].

Description of Event

On September 9, 1998, while operating in Mode 1 at 100% power, the Unit 1 reactor automatically tripped due to a main turbine trip that resulted from a circuit card failure in the solid state protection system (SSPS). Additionally, a trip of both steam generator feed pumps (SGFPs) [SJ] and a main feedwater isolation resulted from the card failure. The plant responded as designed following the event. A root cause analysis was initiated. The cause of the card failure remains under investigation.

Cause of Event

The cause of the trip was failure of universal card A204 in SSPS [JC] train B. Failure of this card has the same effect as 2 out of 3 Steam Generator (SG) level detectors on 1A SG being above the hi-hi level actuation setpoint. This directly caused a main turbine trip, both SGFP trips, and a main feedwater isolation. Since power was above the P-8 interlock setpoint (35%), a reactor trip was directly initiated by the main turbine trip. The failure mechanism for this card is not yet known. The failed card has been shipped to the supplier for analysis. Based on the results of the analysis, additional action will be considered.

Safety Assessment

All safety systems operated as designed and plant response was as expected. The health and safety of the public were unaffected by this event.

This event would not have been more severe had it occurred under different operating conditions.

Corrective Action

The failed card was replaced and both trains of SSPS were tested satisfactorily.

NRC FORM 366A (6-1998) U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET (2)			PAGE (3)					
		YEAR		UEN		REVISION NUMBER			
Joseph M. Farley Nuclear Plant Unit 1	05000348	1998	0	0	4	00	3	OF	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

Additional Information

No similar LERs have been reported by Farley Nuclear Plant in the last two years.

A four hour non-emergency notification was made pursuant to 10CFR50.72.

CATEGORY 1

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

			uclear	98/09/28 NOTARIZEI Plant, Unit 1, Sou		DOCKET # 05000348
STÍNSON	,L.M. Souther	n Nucle	ar One	rating Co.		
MOREY, D	. Souther	n Nucle	ar Ope	rating Co.		
CIID TECT	. TED 00 004 00.0	- 08000	0 *****	!== k=!= c		С
POPOECI				ine trip & conseque		
		replace	ed & U	tor protection sys nit 1 returned to p		A
	980910.WICH 980	926 ILI	•			T
DICTRI	UTION CODE: IE22T	CORTE	DECE	IVED:LTR ENCL	SIZE: A	+ -
				rt (LER), Incident		E
NOTES:						G
	RECIPIENT	COPI	ES	RECIPIENT	COPIES	0
	ID CODE/NAME		ENCL	ID CODE/NAME	LTTR ENC	
	PD2-2 PD	1	1	ZIMMERMAN, J	1 1	R
	122 2 12	-	-	ZIMBRUM, 0		
INTERNAL:	ACRS	1	1	AEOD/SPD/RAB	2 2	Y
	AEOD/SPD/RRAB	1	1	FILE CENTER		
	NRR/DE/ECGB	1	1	NRR/DE/EELB	1 1 1	
	NRR/DE/EMEB	1	1	NRR/DRCH/HICB	1 1	1

NRR/DRCH/HQMB

NRR/DSSA/SPLB

1

1

1

D

U

E

N

T

RES/DET/EIB 1 RGN2 FILE 01 1 ITCO BRYCE, J H EXTERNAL: L ST LOBBY WARD 1 1 1 1 NOAC POORE, W. 1 NOAC QUEENER, DS 1 1 NRC PDR NUDOCS FULL TXT

1

1 1

NRR/DRCH/HOHB

NRR/DRPM/PECB

NOTE TO ALL "RIDS" RECIPIENTS:
PLEASE HELP US TO REDUCE WASTE. TO HAVE YOUR NAME OR ORGANIZATION REMOVED FROM DISTRIBUTION LISTS
OR REDUCE THE NUMBER OF COPIES RECEIVED BY YOU OR YOUR ORGANIZATION, CONTACT THE DOCUMENT CONTROL
DESK (DCD) ON EXTENSION 415-2083

FULL TEXT CONVERSION REQUIRED

TOTAL NUMBER OF COPIES REQUIRED: LTTR 23 ENCL 23